

Current Challenges in Nuclear Safety R&D especially Code Development

R. Kilger, A. Schaffrath, J. Sievers, M. Sonnenkalb, A. Wielenberg
GRS

Challenges Faced by Technical and Scientific Support Organizations
(TSOs) in Enhancing Nuclear Safety and Security

Brussels, Belgium, 15-18 October 2018

Outline

- Introduction
 - Nuclear Energy Perspective in the EU
 - GRS – German TSO & ETSON member
 - Nuclear Code Chain of GRS
- Challenges in nuclear safety R&D / code development
 - TH system behaviour
 - Reactor physics
 - Fuel behaviour
 - Component behaviour
 - Small Modular Reactors
- Conclusions and Outlook

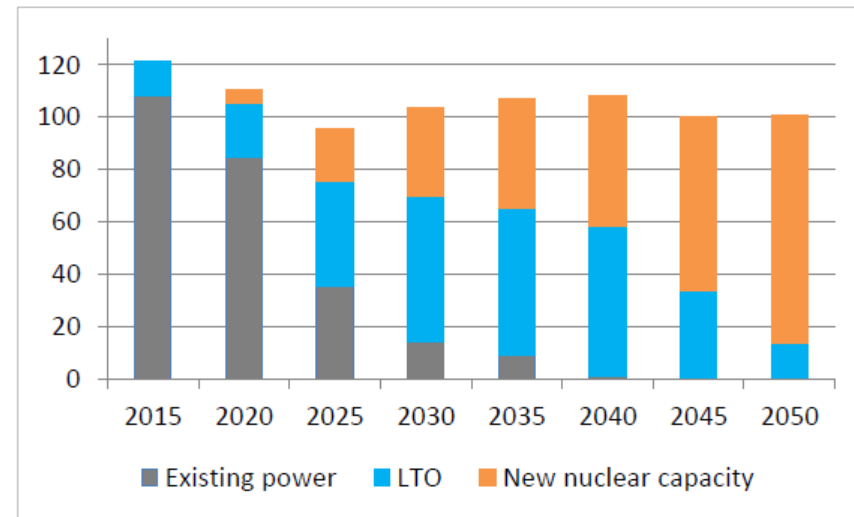
Introduction – Nuclear Illustrative Programme (PINC) of the EU ¹⁾

- Communication of PINC is a requirement under Article 40 of the Euratom Treaty
- Overview of planned investments for next steps of the nuclear lifecycle until 2050
- Nuclear energy is part of the energy mix of half of the EU Member States:
27.5% nuclear, 29.2% renewable sources within the EU ¹⁾
- *EU requires: Member States need to apply the highest standards of safety, security, waste management and non-proliferation as well as diversify nuclear fuel supplies. It will help to achieve the objectives of the 2030 climate and energy framework.*
- Decline in nuclear generation at EU level up to 2025 due to Member States phase out
- Trend would be reversed by 2030 as new reactors are predicted to be connected
- Capacity replacement up to 2050 will most likely be by advanced reactors: EPR, AP 1000, VVER 1200, ACR 1000, ABWR

➤ *Challenges in Nuclear Safety R&D
/ Code Development.*



1) - https://ec.europa.eu/energy/sites/ener/files/documents/nuclear_illustrative_programme_pinc_-_may_2017_en.pdf

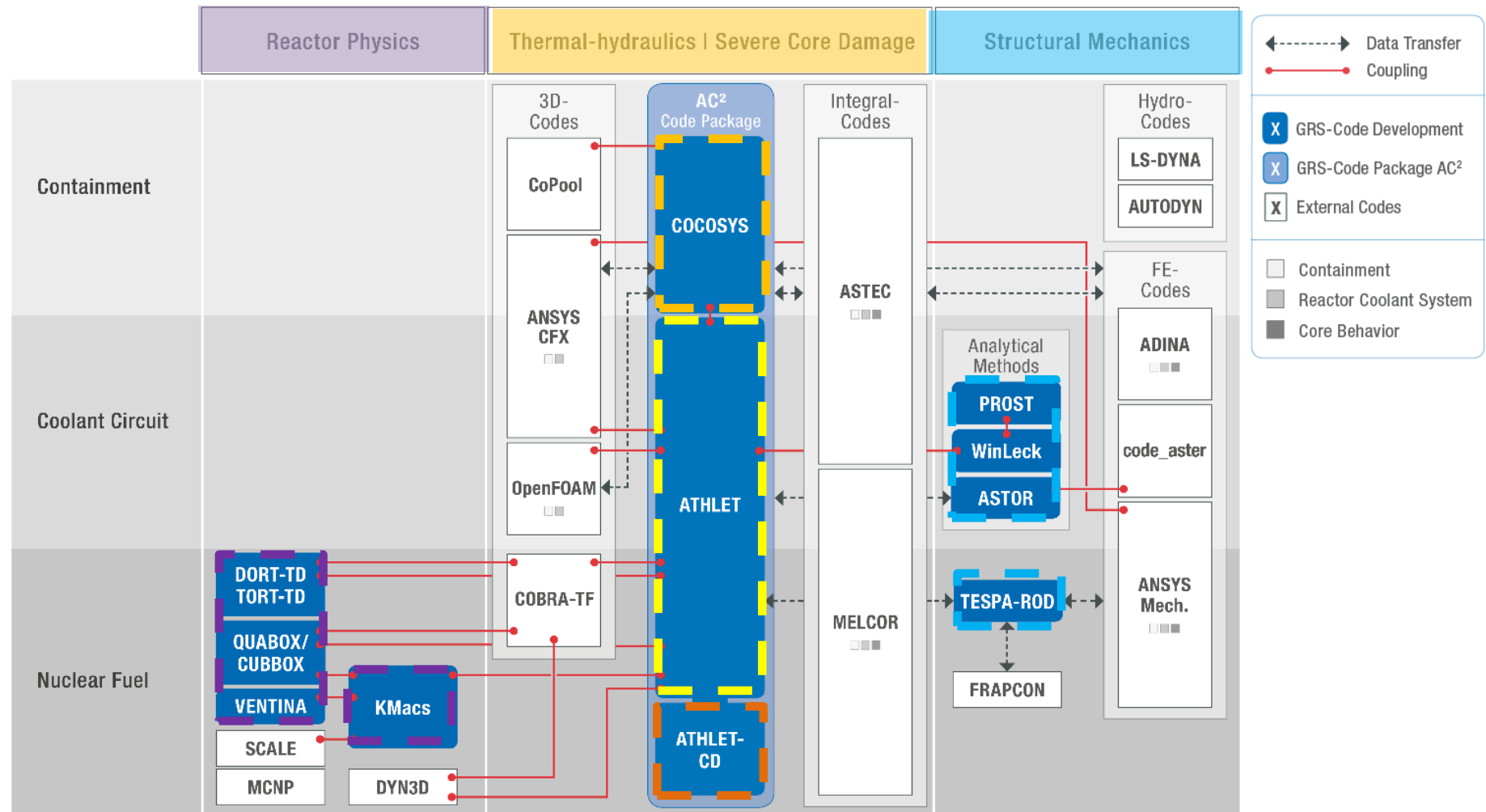


Introduction – GRS, main German TSO & ETSON member

- **GRS** is the main **Technical Support Organization (TSO)** in nuclear safety for German federal government and a major research organization
- GRS is an independent non-profit organization and a Member of ETSON
- GRS is traditionally involved in numerous int. activities e.g. IAEA, OECD/NEA, EU
- Research Tasks and Safety Analyses at GRS are done to provide technical support to different German ministries: BMU, BMWi, AA, BMBF
- GRS as the main German TSO will retain its function in supporting the German government even after phase out of Nuclear Energy production
- **Reactor Safety Research at GRS:**
 - Scientific codes (*Nuclear Code Chain*) – developed, validated and applied for the analyses of operational states, accidents and severe accidents in various NPPs, other facilities, spent fuel storage, etc.
 - GRS Codes represent the current state of science and technology
 - Participation in experimental projects is done to improve understanding of fundamental safety phenomena and to get reliable data for code/model validation (no experimental facilities are operated at GRS)

➤ **Challenges:** *GRS Activities should follow international / EU trends in Nuclear Energy.*

Introduction – GRS' Nuclear Code Chain



➤ **Challenges:** Further development / validation of Nuclear Code Chain, so that it can be applied for safety analyses of existing / advanced / innovative NPPs & SMR.

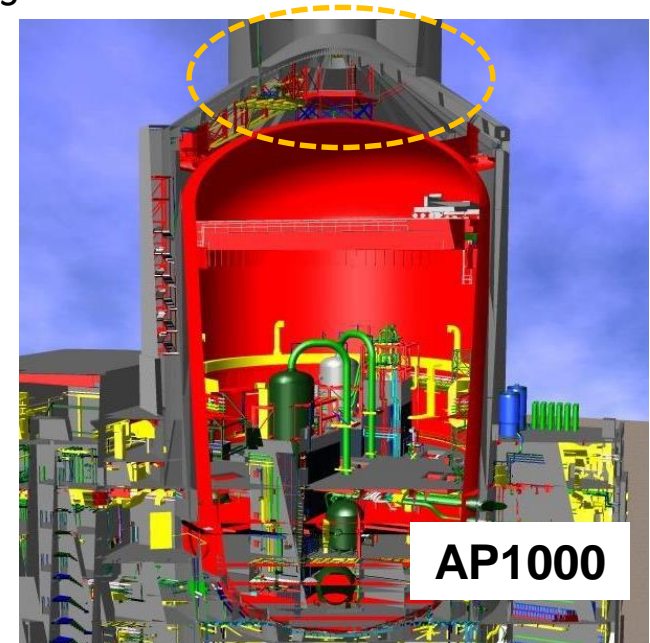
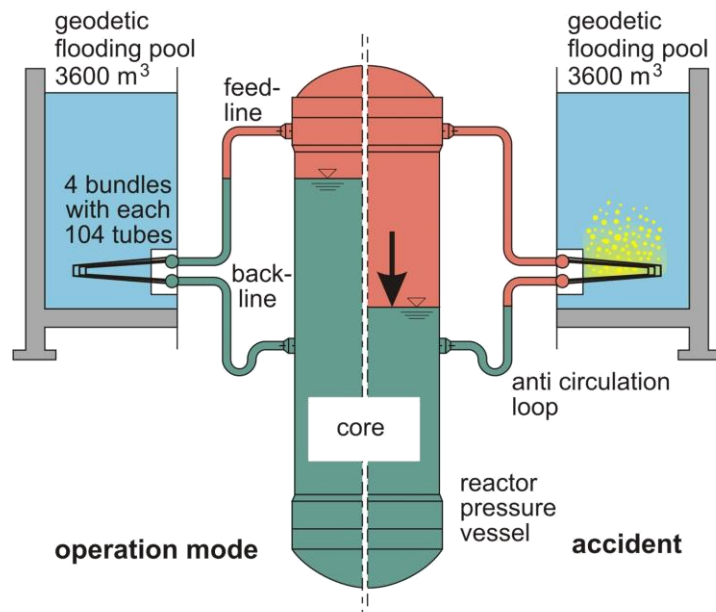
Challenges in Code Development – Thermal Hydraulics

- GRS TH code chain was originally developed for Gen. II reactors

➤ **Challenges:** *Application for Gen. III/III+ and Advanced Designs:*

- *Passive cooling systems (convection and condensation phenomena, interactions cooling circuit – containment, etc.), ultimate heat sink (heat pipes etc.)*
- *Material properties, specific correlations and models relevant for Accelerator Driven Systems and Gen. IV (near vacuum, supercritical water, liquid metals e.g. sodium or lead-bismuth, etc.)*

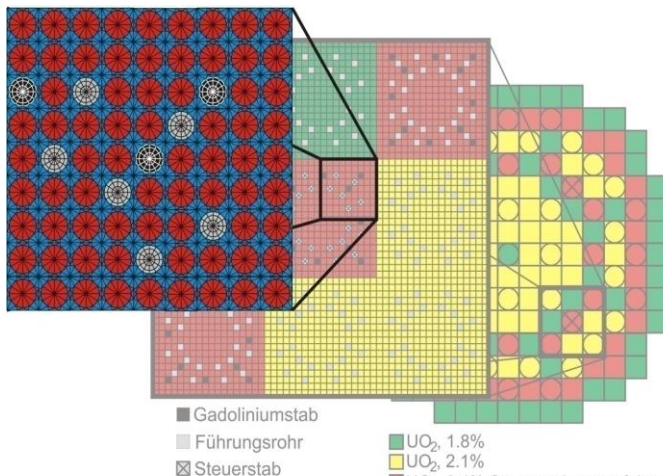
Passive cooling systems



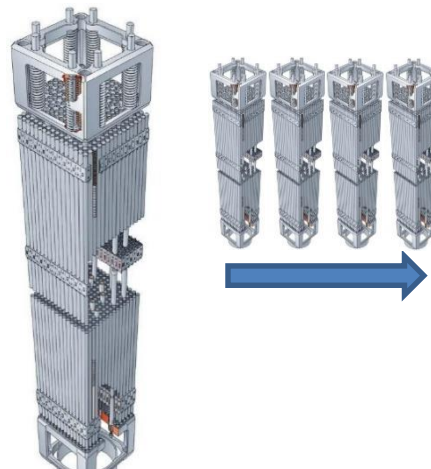
Challenges in Code Development – Reactor Physics

- Different core simulators have been developed:
 - **Assembly wise Core Simulator** (nodal) for calculation of power distribution (QUABOX/CUBBOX using diffusion approximation)
 - **Pin-by-pin Core Simulator** for improved calculation of rod wise power distribution (DORT-TD/TORT-TD – multigroup transport), especially to describe dedicated phenomena in innovative reactor systems
 - New **Core Simulator KMacS** for detailed operation cycle calculations, applied for western-type PWR

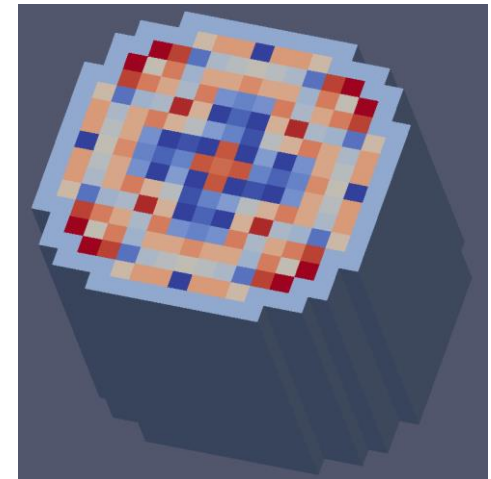
Transition from Coarse-Mesh to Pin-by-Pin



Cross-Section Generation



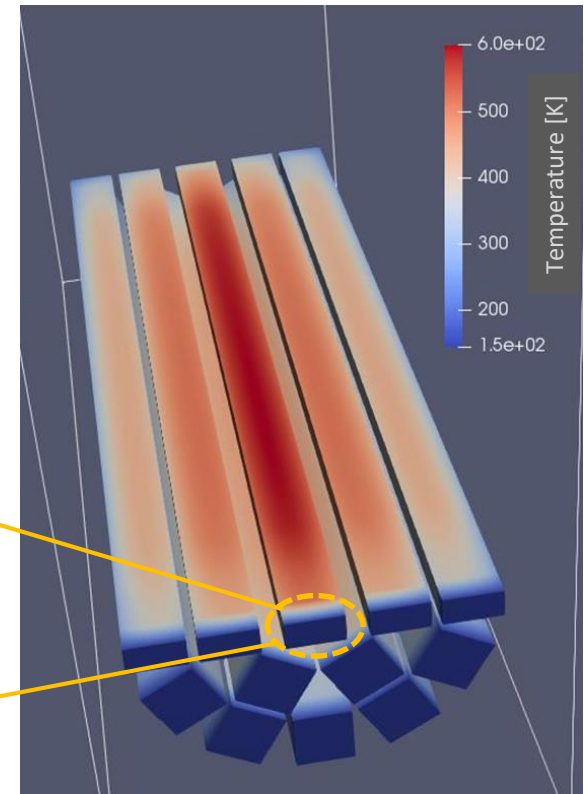
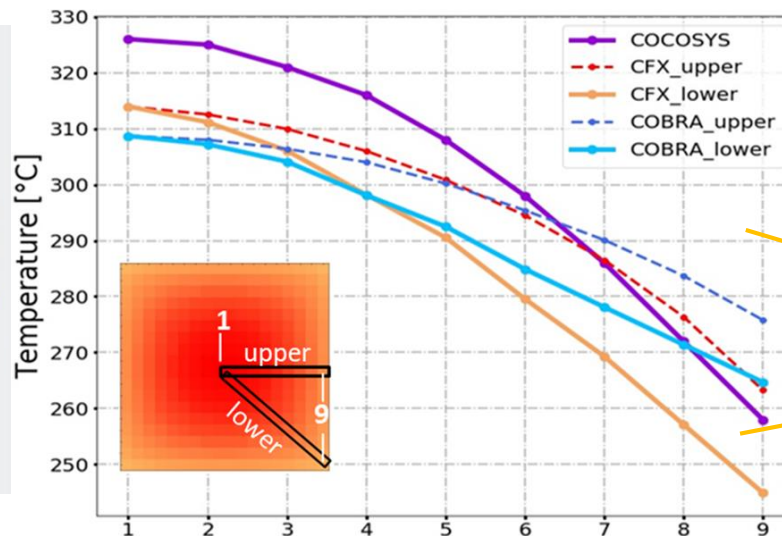
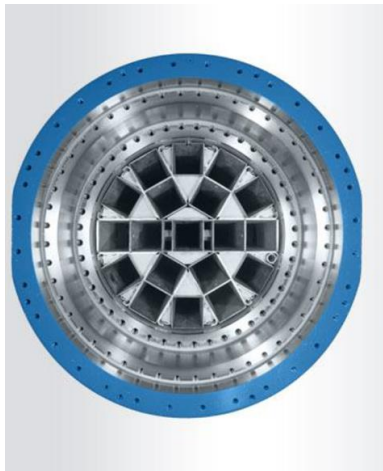
Cycle Calculation



➤ **Challenges:** Extension of core simulators for other fuel assembly geometries and multi-physics / multi-scale code coupling in the field of neutron kinetics.

Challenges in Code Development – Fuel behaviour / storage

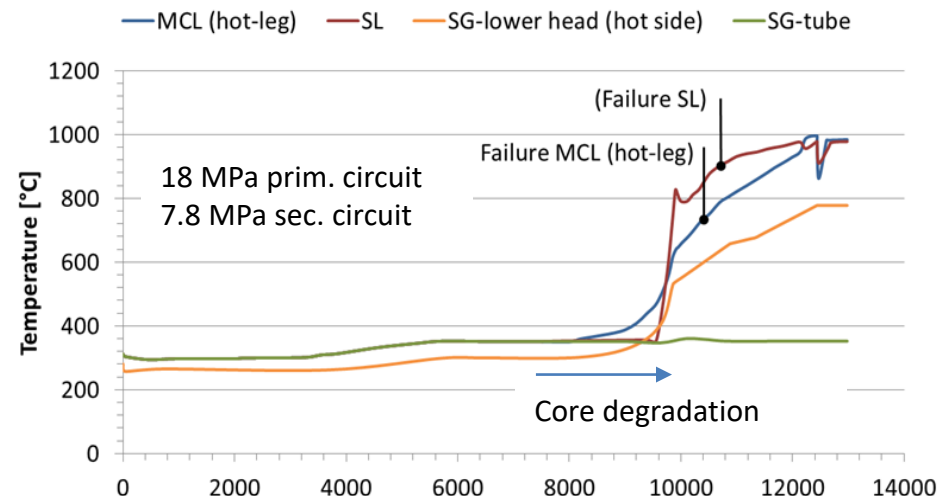
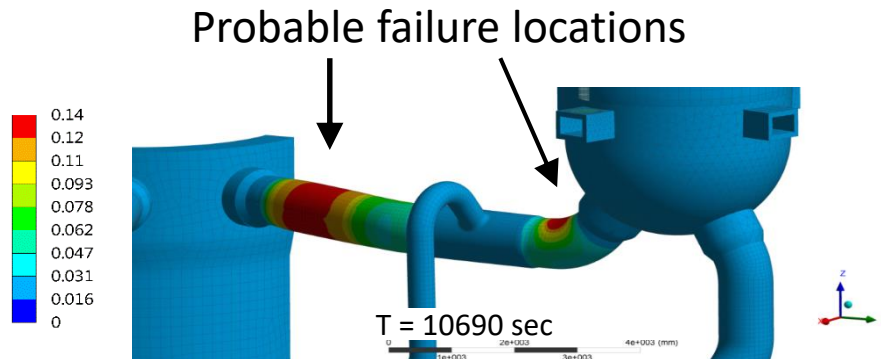
- Spent fuel dry cask interim storages are currently commissioned for ~40 years
- Extended long-term operation of such dry storage facilities is expected
- 3D temperature field of a loaded cask was analysed using different codes:
COBRA-SFS, COCOSYS, ANSYS CFX
 - Large variations of the cladding temperatures can be observed in both horizontal and vertical directions
 - The temperature results are used to predict the thermo-mechanical cladding behaviour



➤ **Challenges:** Additional knowledge and data about material and component performance in conjunction with predominant conditions are necessary, etc.

Challenges in Code Development – Component behaviour

- Development / Validation / Application of Structural Mechanics codes for analyses of component behaviour under severe accident loads – coolant circuit & containment
- Calculation of component failure time with ASTOR, ANSYS Mechanical
- Example: Station-Blackout-Scenario with core melt in a PWR
 - Progressive failure process
 - Strong increase of temperature leads to radial deformations and strains within short time up to failure due to plastic instability
 - First failure: Main Coolant Line (hot-leg)



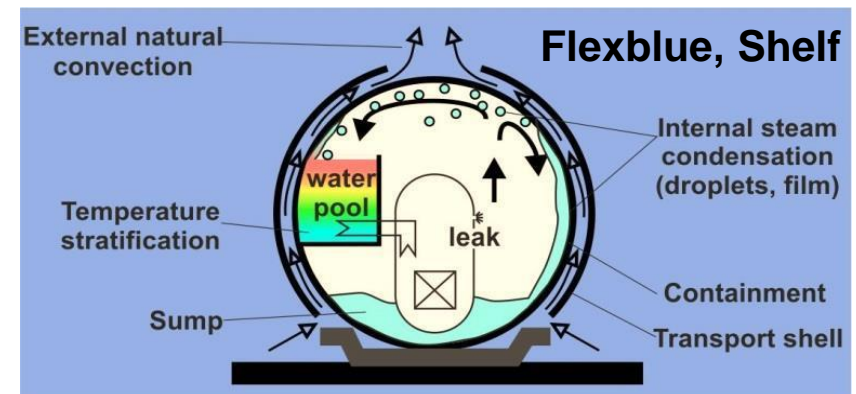
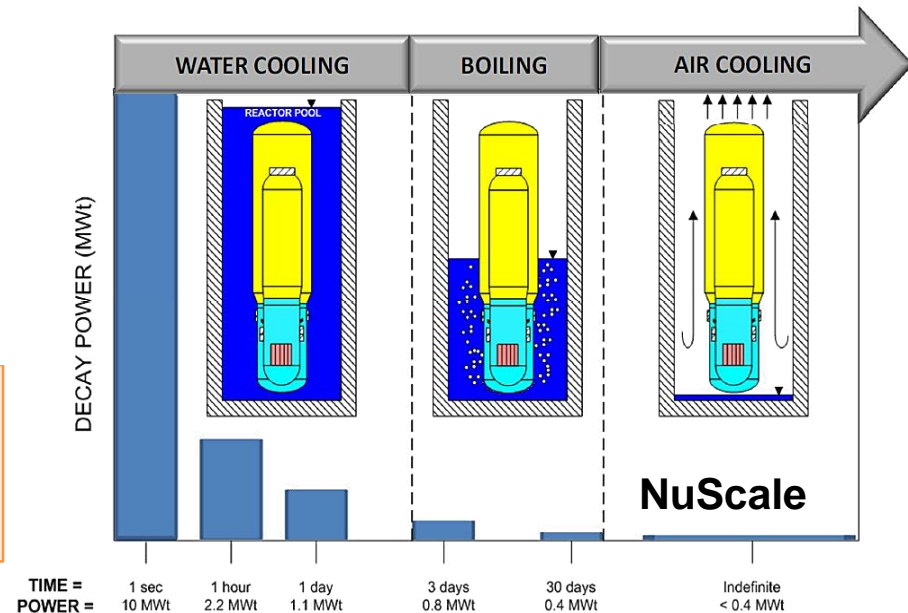
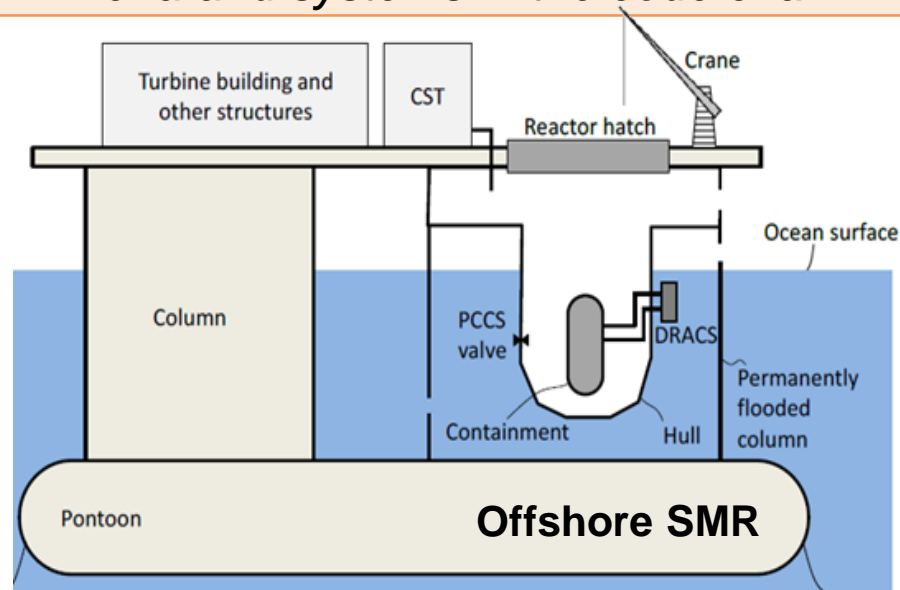
➤ **Challenges:** Improvement of methods / codes related to material conditions under high temperature / pressure, consideration of local effects, asymmetric loads in combination with determination of finite leak size, etc.

Challenges in Code Development – Small Modular Reactors (SMR)

>130 SMRs are running, >100 new SMR concepts are under development:

- Various containment concepts are realized
- Integrated, often self-pressurized passive cooling systems with innovative heat exchanger designs (heat pipes, etc.)

➤ **Challenges:** SMR require updates in the simulation of new physical phenomena and systems in the code chain.



Conclusions and Outlook

- The German government decided to terminate the use of nuclear energy production by the End of 2022.
- Renewable energy production is an increasing factor in Germany.
- GRS as the main German TSO will retain its function in supporting the German government in all relevant issues on nuclear safety and radioactive waste management in Germany and abroad.
- Research at GRS in reactor safety so far was and will be focused on code development and validation for existing / advanced / innovative NPPs and SMR.
- Various *Challenges* have been identified; some examples have been discussed.
- Other *big Challenges* are
 - the conservation and transfer of “nuclear knowledge” to the next generation of nuclear engineers and physicists and
 - the restructuring / modernisation of our “legacy codes”.
- *GRS will have the necessary staff, competences, know-how and validated tools for safety assessments for Advanced NPPs and SMR in future as well.*

Thank you very much for your attention!

Acknowledgement: The development of the GRS nuclear simulation chain is currently funded by the German Federal Ministry for Economic Affairs and Energy (BMWi) within several projects (essentially RS1507, RS1518, RS1520, RS1526, RS1532, RS1535, RS1538, RS1543, RS1546, RS1547).

Supported by:



Federal Ministry
for Economic Affairs
and Energy

on the basis of a decision
by the German Bundestag